



University of Pittsburgh

ME/ENGR 2100

Fundamentals of Nuclear Engineering

Fission Reactor Basics:
Criticality Control

Stephen R. Tritch Program in Nuclear Engineering
Swanson School of Engineering
University of Pittsburgh





Relevant Reading Assignments

- Sections 6.5 to 6.8 of “*Introduction to Nuclear Engineering*” by Lamarsh and Baratta, 3rd Edition.
- Chapter 3 of “*Nuclear Reactor Analysis*” by Duderstadt and Hamilton
- Page 100-120 of “*Nuclear Engineering: Theory and Technology of Commercial Nuclear Power*” by Knief, 2nd Edition.



Relevant Reading Assignments

- “Secrecy, simultaneous discovery, and the theory of nuclear reactors” by Spencer Weart. American Journal of Physics, Vol. 45(11). November 1977



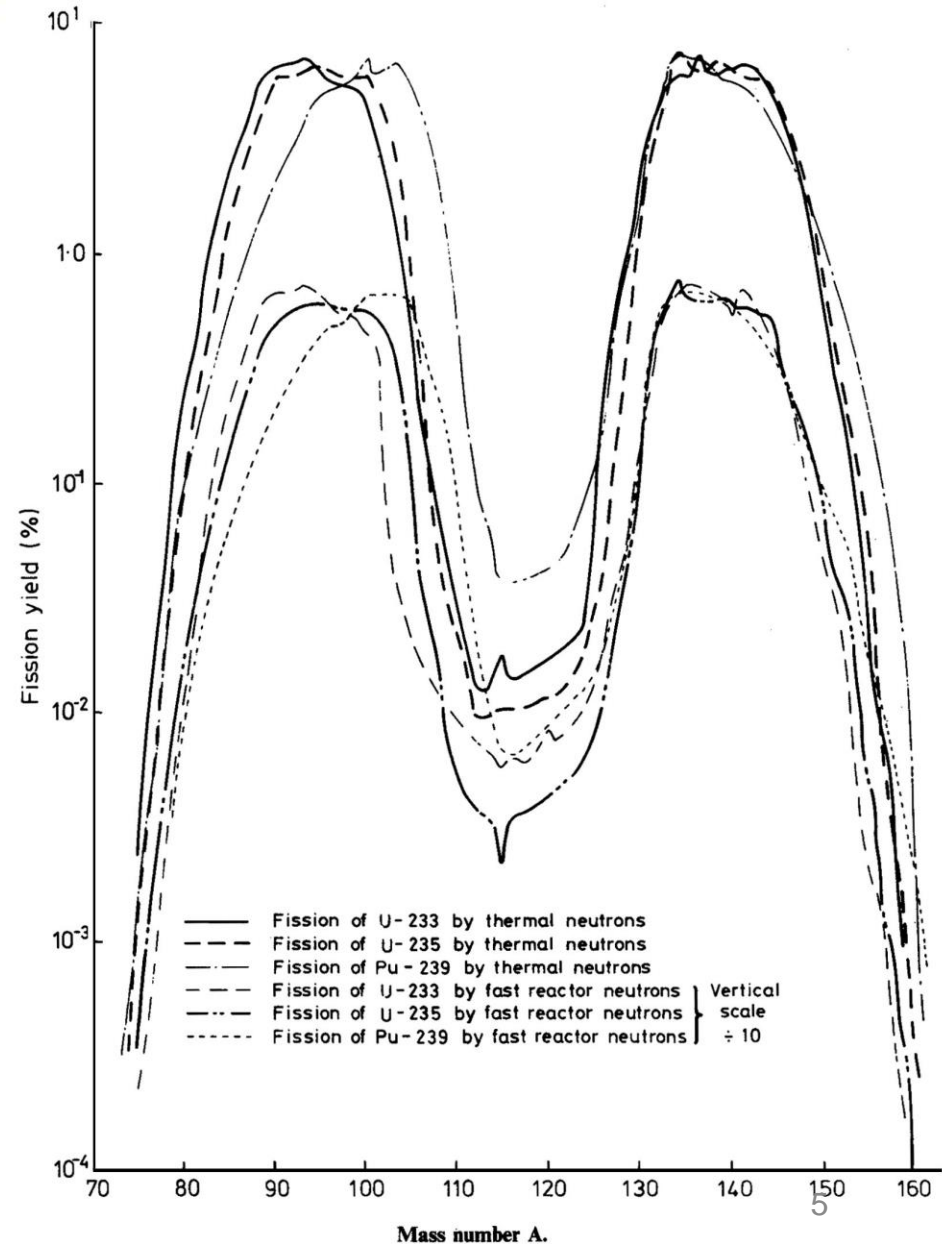
Learning Objectives

- Explain how the terms in the four and six factor formulas may be adjusted to control criticality in reactor and processing settings



Criticality Control

- No reactor can be constantly critical
- Fuel depletion
 - Fission removes a fuel atom and creates two new atoms
 - Transient fission product poisons
 - Xenon and Samarium
 - Fission product poison build up
 - ^{83}Kr , ^{95}Mo , ^{143}Nd , ^{147}Pm
- Temperature (moderator density) changes





Creating Neutron Balance

- States of criticality
 - $k_{eff} = 1$ Critical
 - $k_{eff} > 1$ Supercritical
 - $k_{eff} < 1$ Subcritical
- In order to keep an operating nuclear reactor critical we will need to “adjust” terms in the neutron balance
- Neutron balance controls
 - Production
 - Absorption
 - Leakage



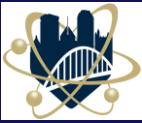
Creating Neutron Balance

- Let us consider how we could adjust these parameters to achieve a target k_{eff} for two different applications
- Nuclear Power Plant
 - Target k_{eff} :
 - $k_{eff} = 1$ for steady-state operation
 - $k_{eff} > 1$ for start-up, $k_{eff} < 1$ for shutdown
- Nuclear Fuel Processing Facility
 - Target k_{eff} :
 - $k_{eff} < 1$ under all possible conditions (including accidents)



Neutron Balance (Reactor)

- Reactor Criticality Requirements
 - Operation Modes
 - Power Reactors (Startup / Steady-State / Shutdown)
 - Some Research Reactors (Pulse Mode)
 - All reactors have emergency shutdown (SCRAM) capability
 - Routine adjustments to reactor criticality are required
 - Account for power fluctuations and feedback effects
 - Fuel depletion, density changes of moderator
 - Small frequent adjustments: control rods (in PWR)
 - Larger, planned, adjustments: soluble boron (in PWR)
 - BWR reactors use control rods and coolant flow feedback to adjust criticality.



Neutron Balance (Reactor)

- Nuclear Reactor
 - **Production**
 - Determined by the total fissile content of the core.
 - Initial fuel loading.
 - Conversion of fertile nuclides (breeding).
 - Refueling
 - On-line or Shutdown



Neutron Balance (Reactor)

- Nuclear Reactor
 - **Production**
 - For a modern commercial PWR core:
 - Ceramic UO_2 pellets using Uranium that has been enriched to 3-5 wt % ^{235}U .
 - 10-30% of power produced in a PWR is due to Plutonium bred in the reactor
 - Currently reactors operate 18-24 months between refuelings



Neutron Balance (Reactor)

- Nuclear Reactor
 - **Absorption**
 - Cladding, Structure, Coolant
 - Control Rods
 - Soluble Poisons
 - Burnable Poisons
 - Fission-Product Poisons

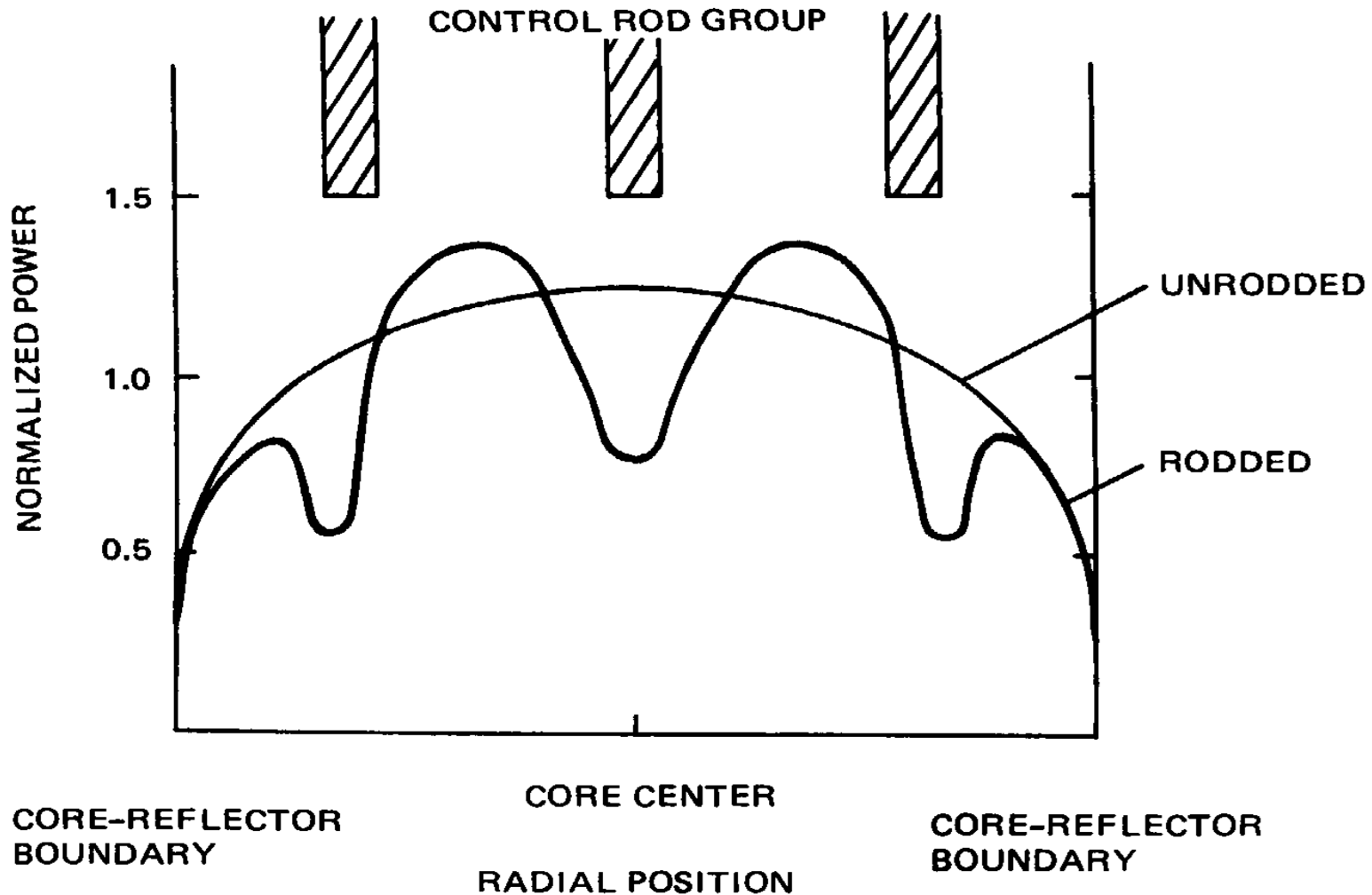


Neutron Balance (Reactor)

- Nuclear Reactor
 - **Absorption**
 - Modern reactor designs
 - Moveable control rods (CR) to change power level and maintain steady state operation.
 - Movable safety rods (SR) to quickly shut down reactor and ensure $k_{\text{eff}} < 1$.
 - Soluble boron in reactor coolant (PWR only) to “shim” k_{eff} (like trim control in an airplane).
 - Fixed burnable poisons (boron or gadolinium) that deplete during operation.

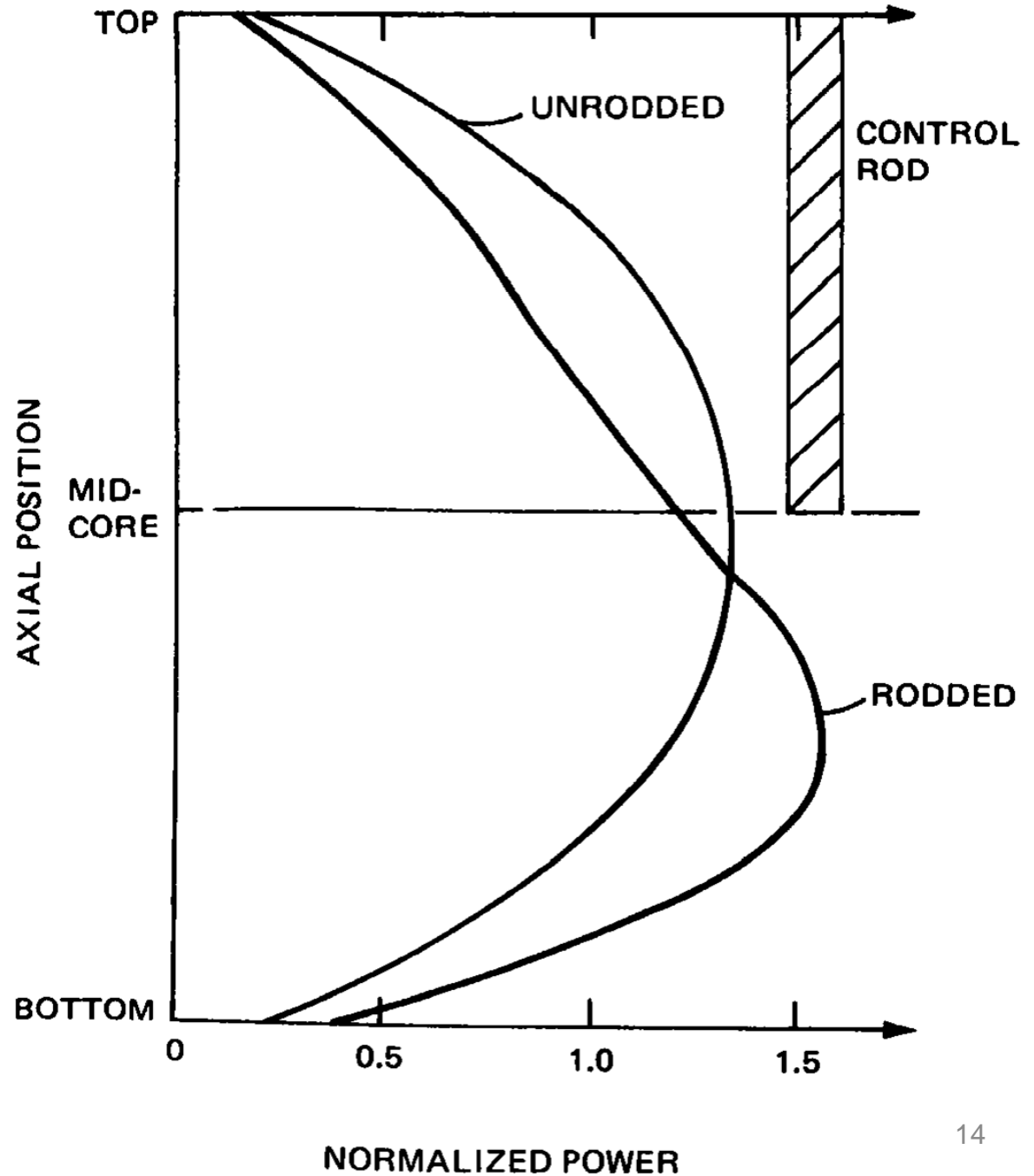


Radial Flux w/ Control Rods





Axial Flux w/ Control Rods





Neutron Balance (Reactor)

- Nuclear Reactor
 - **Leakage**
 - Core size and shape
 - “Reflection” of neutrons back into the core
 - Density of core material(s)
 - (temperature-dependent)

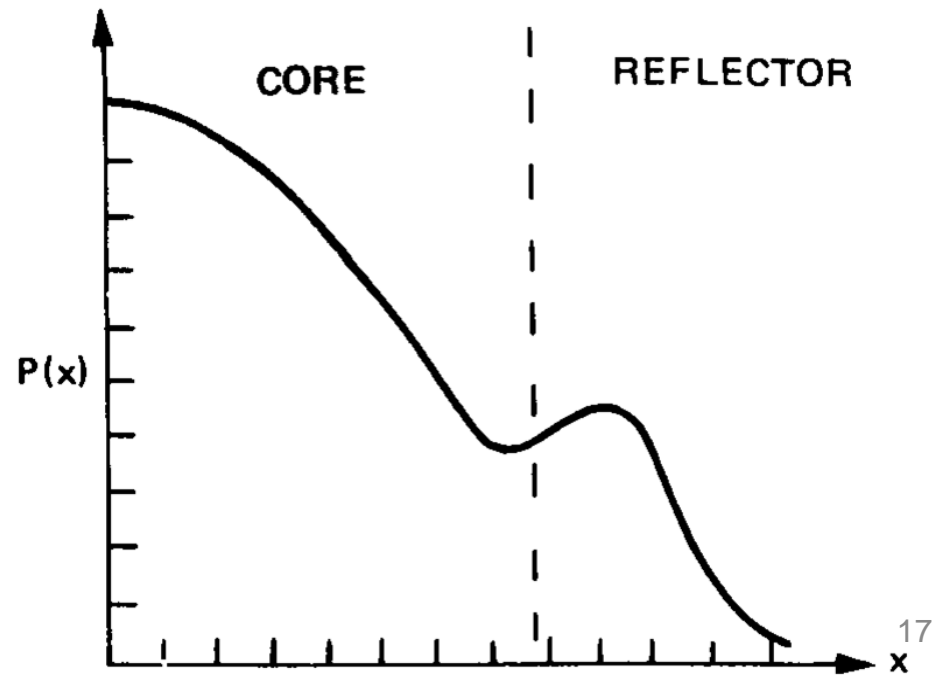
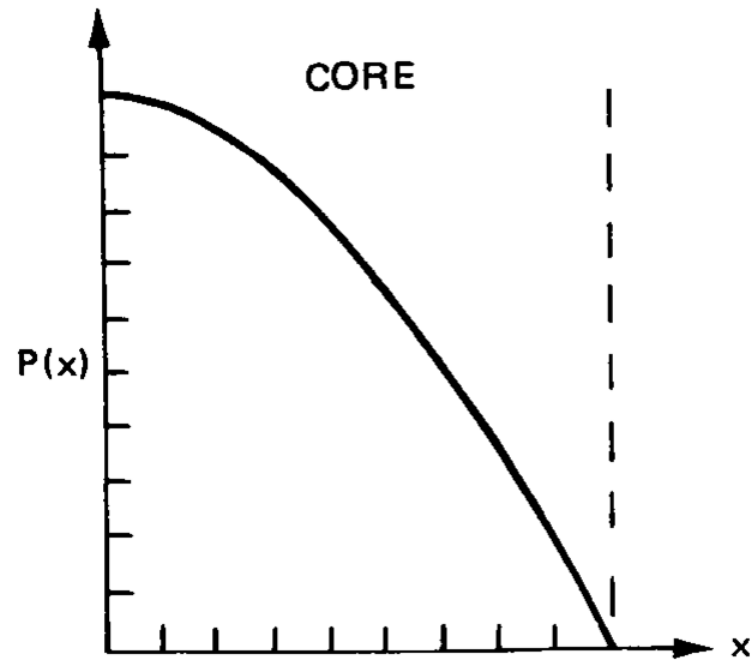


Neutron Balance (Reactor)

- Nuclear Reactor
 - **Leakage**
 - Primarily determined by reactor design
 - Modern reactor designs:
 - Use a cylindrical core shape to reduce surface-to-volume ratio while still allowing easy access to fuel
 - Include a material (usually water) surrounding the core to reflect escaping neutron back into the active fuel region of the core



Water-Reflector Effect on Minimum Core Size





Neutron Balance (Reactor)

- Nuclear Reactor
 - ***Moderation***
 - Controls how effectively neutrons can slow down to thermal energies.
 - Determined by selection of moderator material and pin dimensions (diameter and pitch)



Neutron Balance (Reactor)

- Nuclear Reactor

- ***Moderation***

- In modern PWR designs

- Light water (H_2O) is used as both moderator and coolant
 - Pin diameter: ≈ 1 cm
 - Pin pitch: ≈ 1.25 cm



Neutron Balance (Fuel Facility)

- Fuel Facility
 - Always Subcritical!
 - Designed subcritical, no required adjustments
 - Verify that current configuration is subcritical
 - Confirm that proposed changes will be subcritical
 - $k_{\text{eff}} < 1$ under all conditions
 - Must account for:
 - » Uncertainties in Experimental Data and Calculations
 - » Normal, Anticipated Abnormal & Credible Accident Scenarios



Criticality Control (Fuel Facility)

- Fuel Facility
 - **Production**
 - Determined by the total fissile content present in a single location.
 - Must consider the mass, enrichment, and concentration (density) of fissile materials.
 - Fuel handling sites have strict limits on these quantities to prevent accidents.



Neutron Balance (Fuel Facility)

- Fuel Facility
 - **Absorption**
 - Non-Fissile Materials (Enrichment)
 - Non-Fissionable Materials
 - Solid Poisons
 - Soluble Poisons



Neutron Balance (Fuel Facility)

- Fuel Facility
 - **Leakage**
 - Density
 - Favorable Geometry
 - High Leakage (Long Cylinder/Thin Slab)
 - Individual Unit Subcritical
 - Reflection
 - Separation of Units



Neutron Balance (Fuel Facility)

- Fuel Facility
 - **Moderation**
 - Affects
 - Production
 - Absorption
 - Leakage
 - Important factor in complex interactions between:
spacing, absorption, and moderation among units



Beyond 4 & 6 Factor Formulas

- The four factor formula gives us a good way to examine the competing bulk neutron balance effects in an infinite system.
- In finite systems, neutron leakage must be included when determining the overall neutron balance for the system.
- When designing critical systems (reactors) we need to know the spatial distribution of neutrons in the core, in addition to the multiplication factor.



Beyond 4 & 6 Factor Formulas

- In a large reactor, different parts of the core may be behaving very differently
 - Outer regions of the core will have a large amount of neutrons escaping from the core, and will be losing a large fraction of neutrons than are born within the region.
 - Neutrons produced in inner regions of the core have little chance of escaping the core. These inner regions will effectively produce more neutrons than are needed locally for fission.
- For the reactor as a whole to be critical, these local regions must balance each other out.
 - Different concentrations of neutron densities and reaction rates throughout the core.
 - Neutrons “flow” from the center of the core towards the edge.



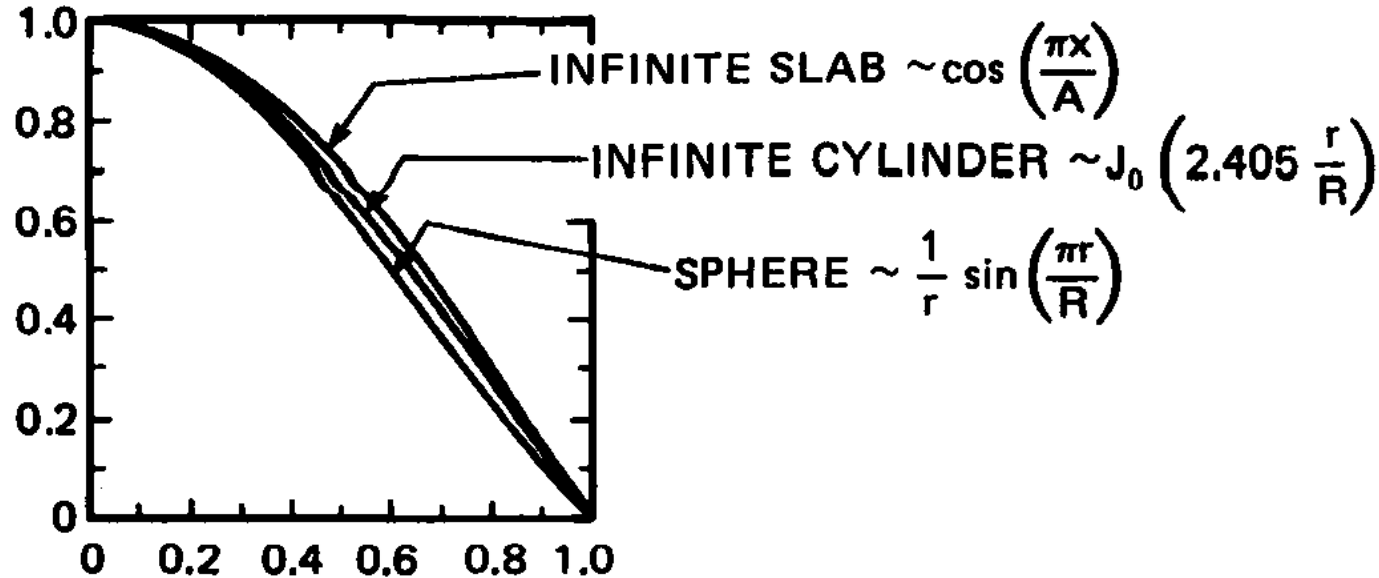
Neutron Density

- During steady-state operations there is a natural spatial distribution of neutrons throughout the core.
- This natural distribution depends on the shape of the reactor and the locations of fissile fuel and neutron poisons in the core
 - Peaked in center
 - Low near edge of core
 - Low density near neutron poisons
- In addition to the gross shape of the neutron density, there are local variations that can have a significant effect on the behavior of the core
 - Localized peaking is usually limiting condition in core



Neutron Densities by Geometry

Normalized Neutron Density

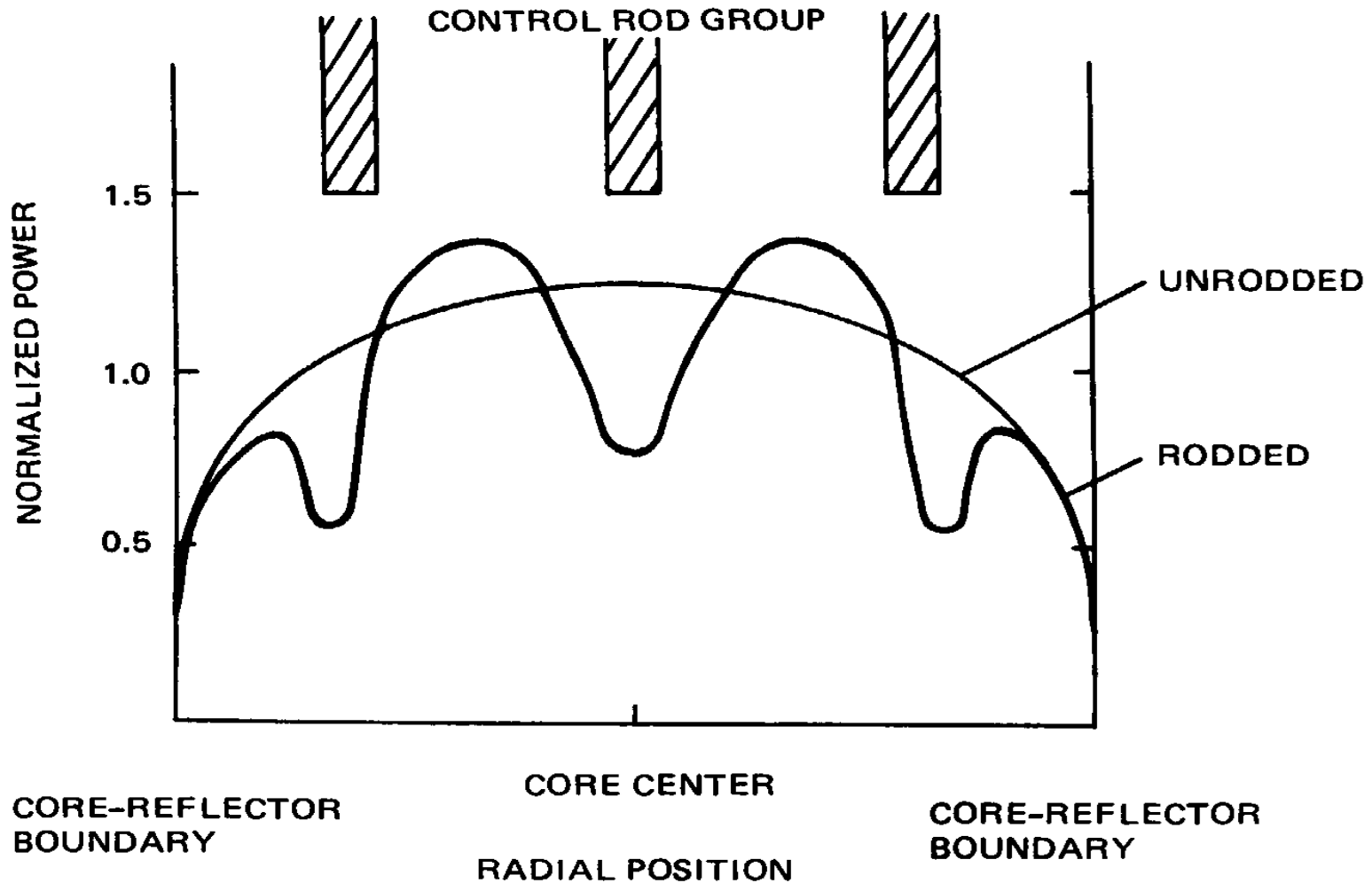


Distance from Core Center (Radius)

- All Three Neutron Densities are “Cosine-Like”
- Peaked in the Center
- Zero at the Edges

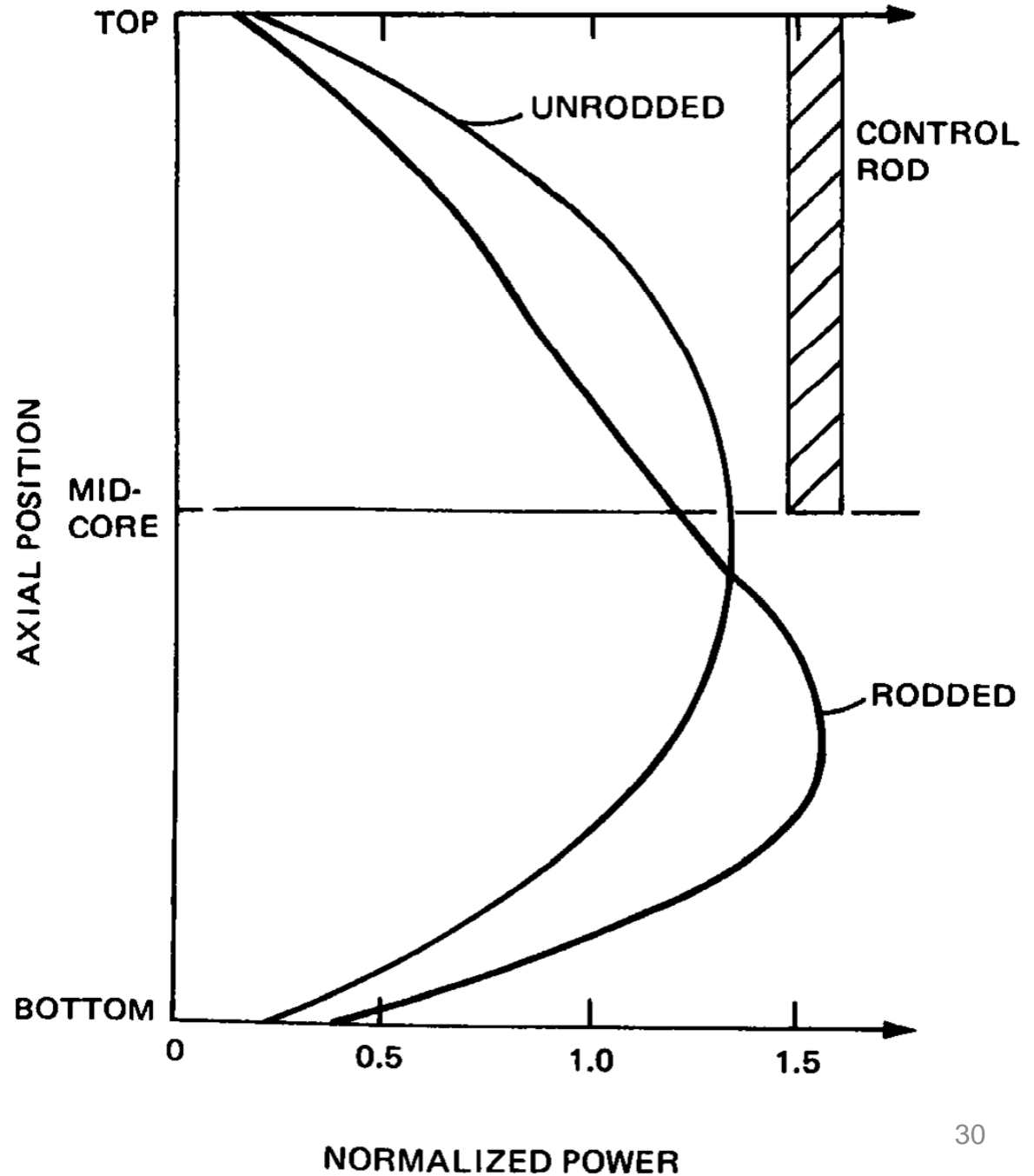


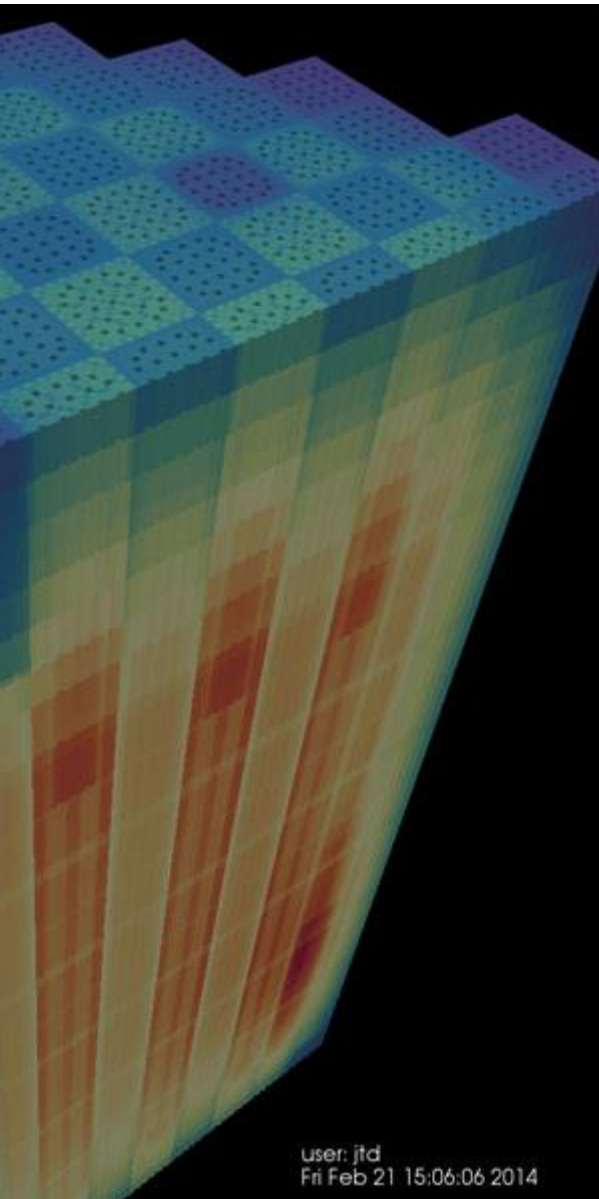
Radial Neutron Density





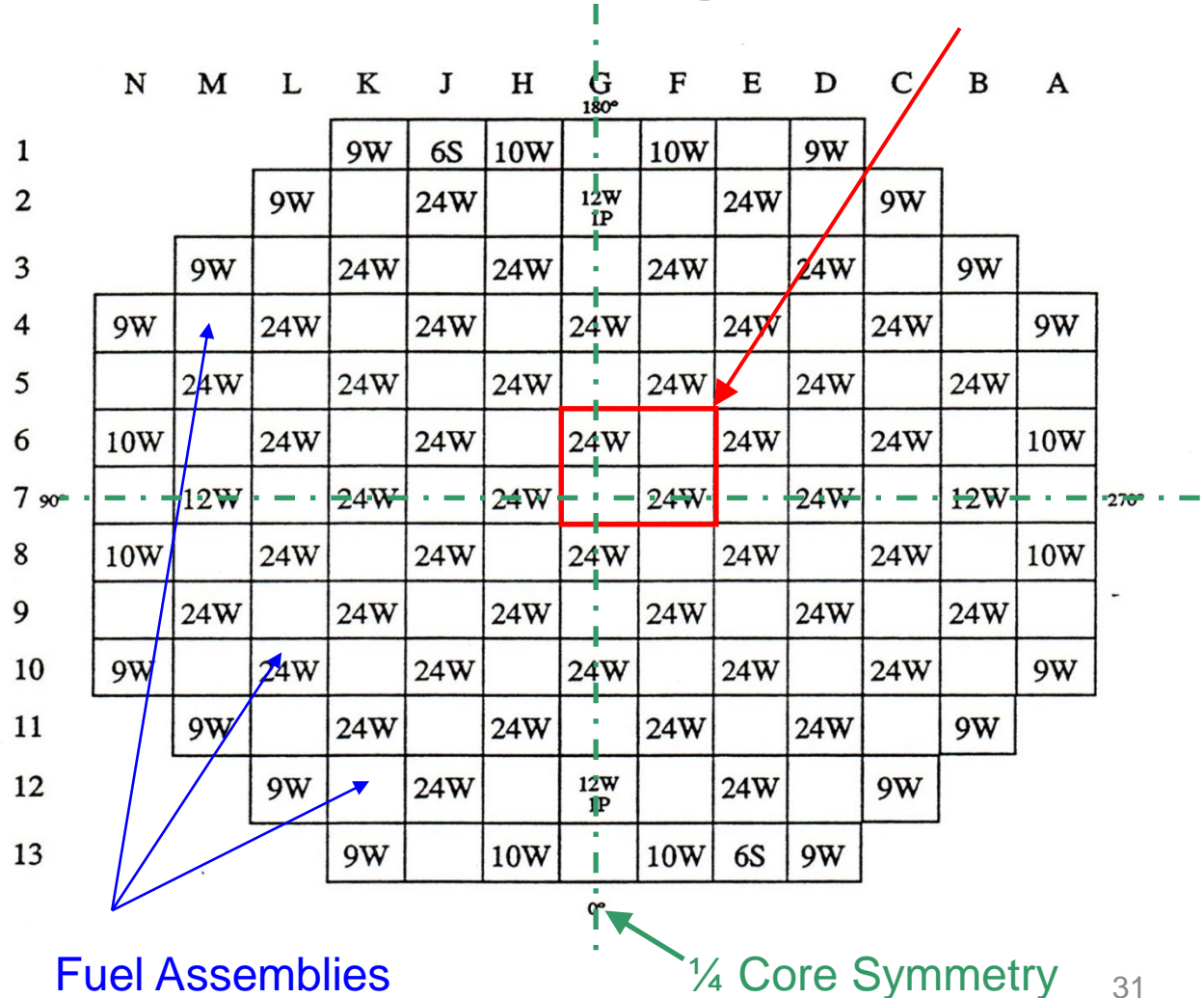
Axial Flux w/ Control Rods





AP600 Core Design

“Zoom In” on these 4 assemblies



Fuel Assemblies

1/4 Core Symmetry

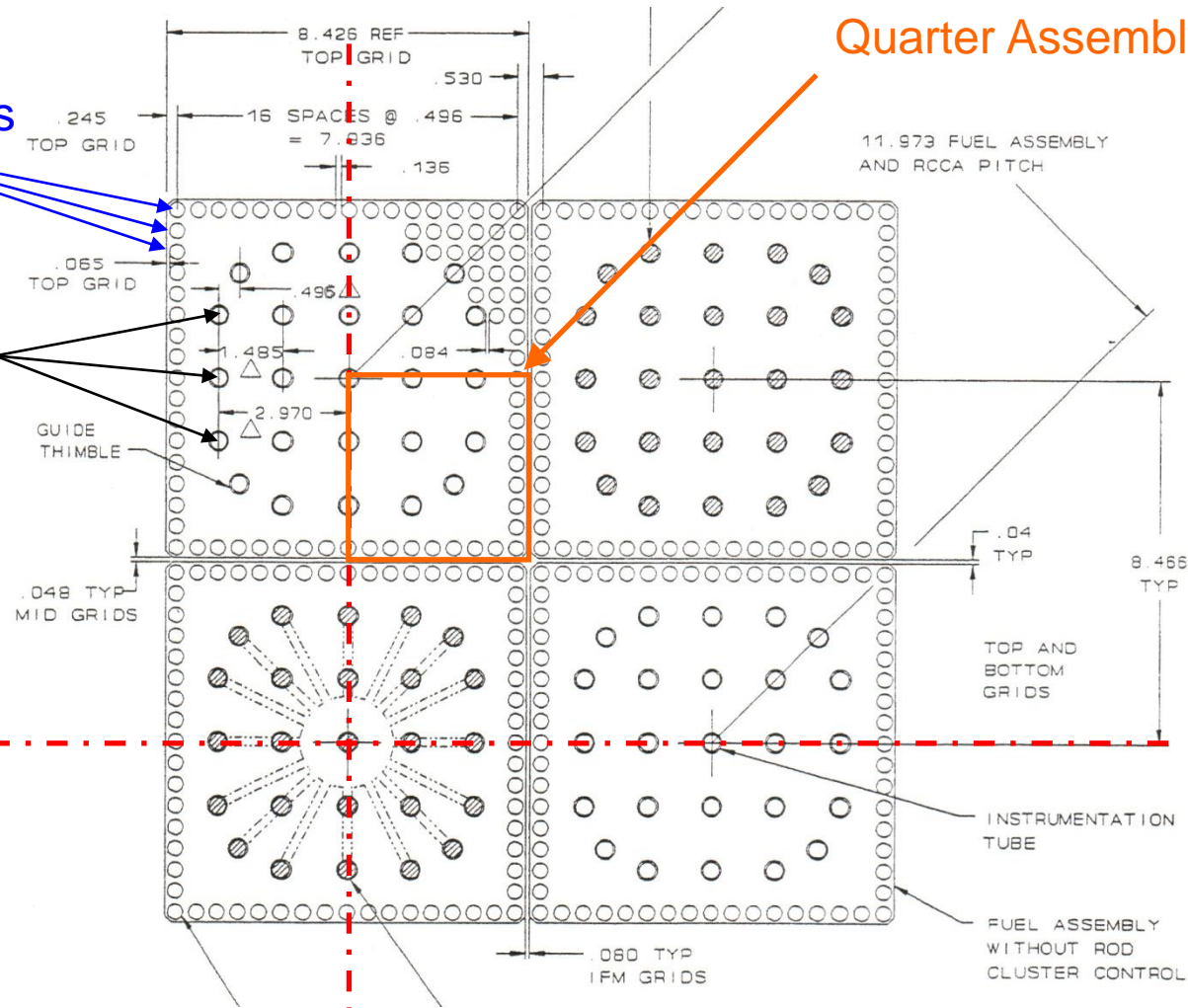
AP600 Assembly Design

Zoom In on
Quarter Assembly

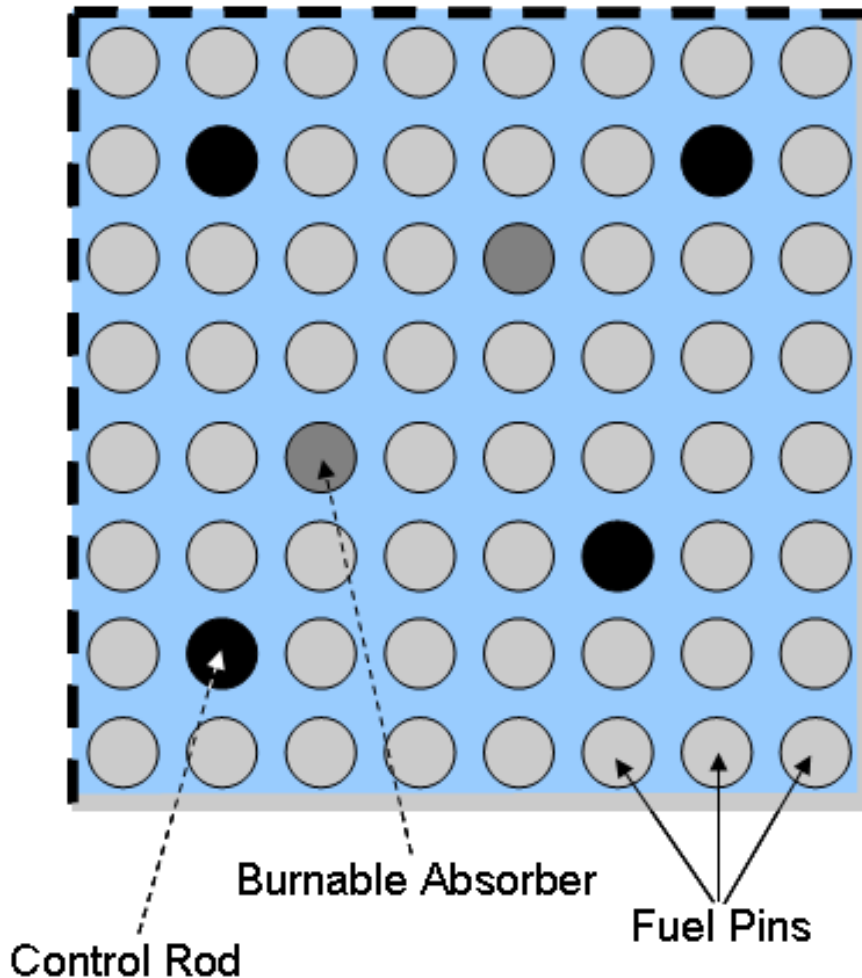
Fuel Elements

Control Rods

¼ Assembly
Symmetry



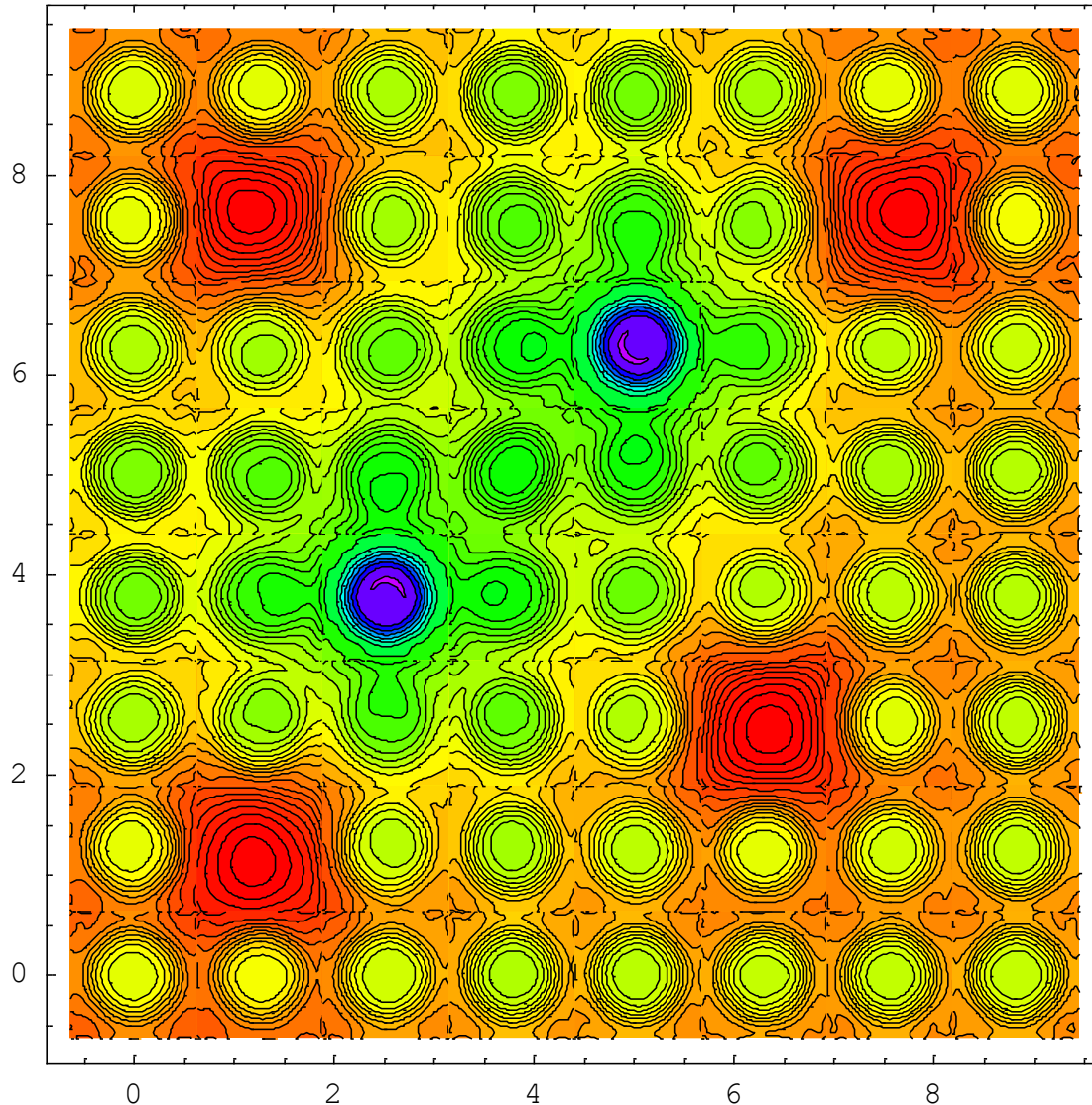
Simplified AP600 Assembly Model



- Simplified 2-D model of an AP600 quarter assembly.
- Contains UO₂ fuel, boron control rods, and B₄C burnable absorber rods.
- Reflecting boundary conditions on all sides.



Neutron Density, Control Rods Out

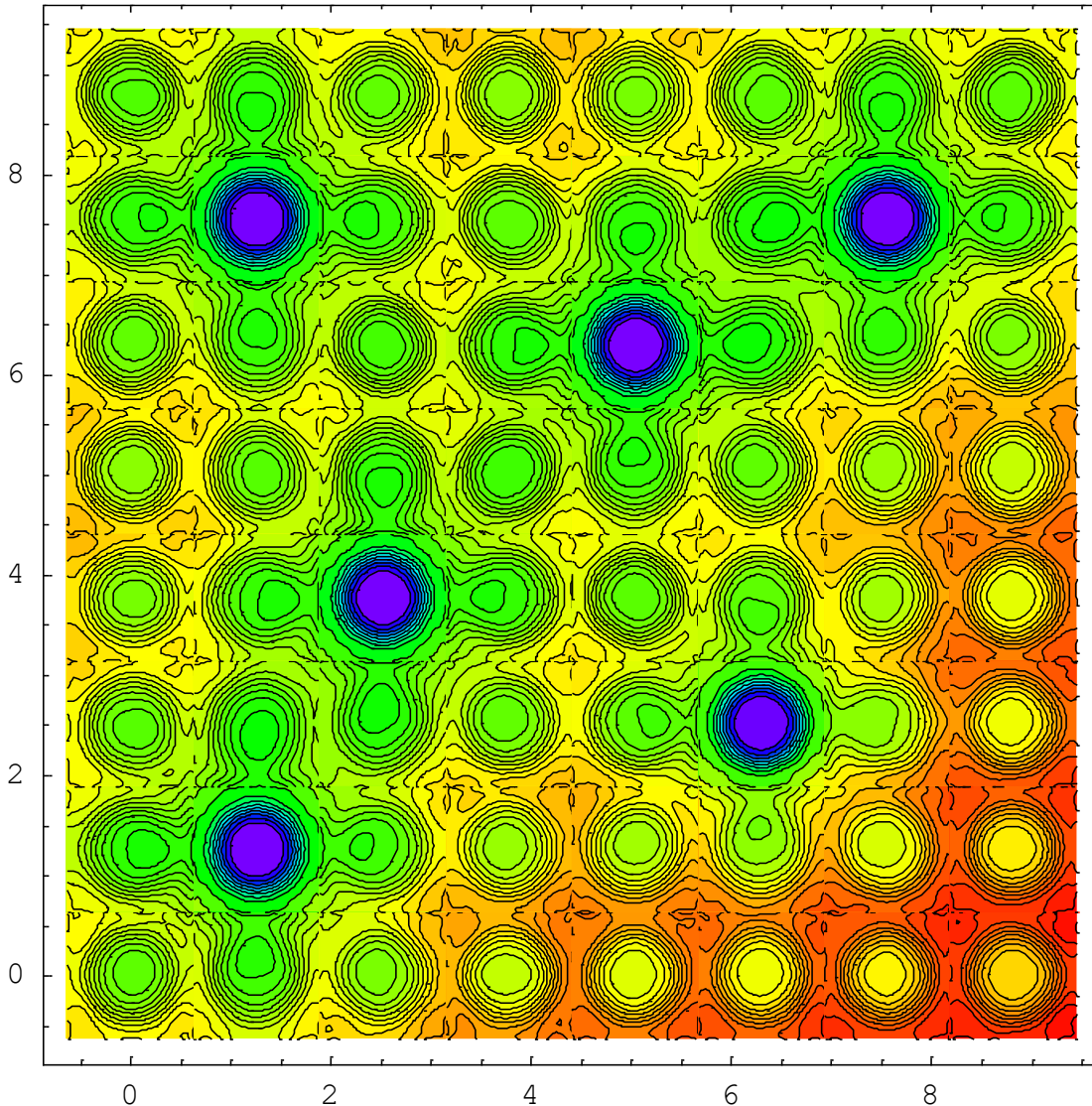


$$k_{\text{eff}} = 1.1630$$

- Notice the local variations in neutron density.
- Different elements can have different neutron densities.



Neutron Density, Control Rods In

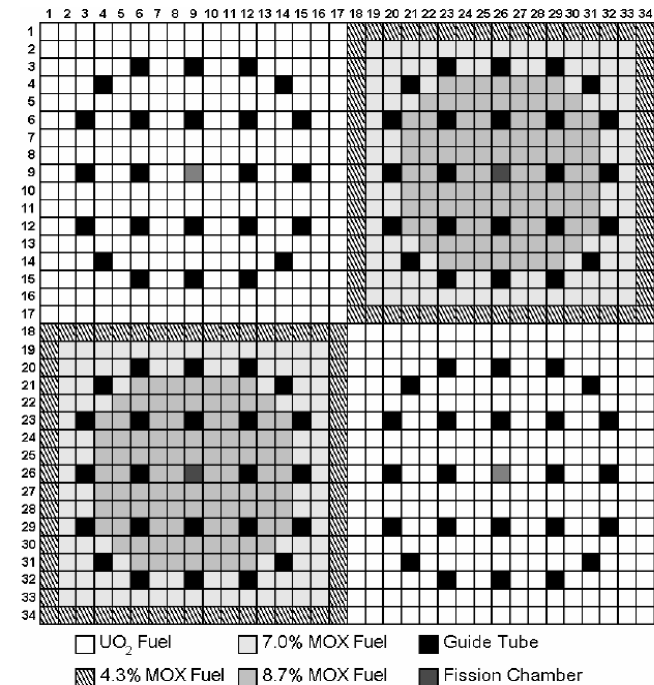
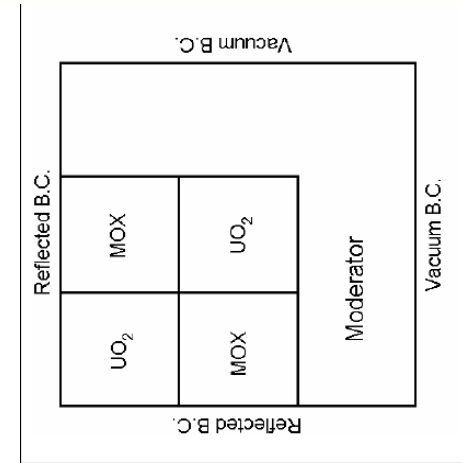
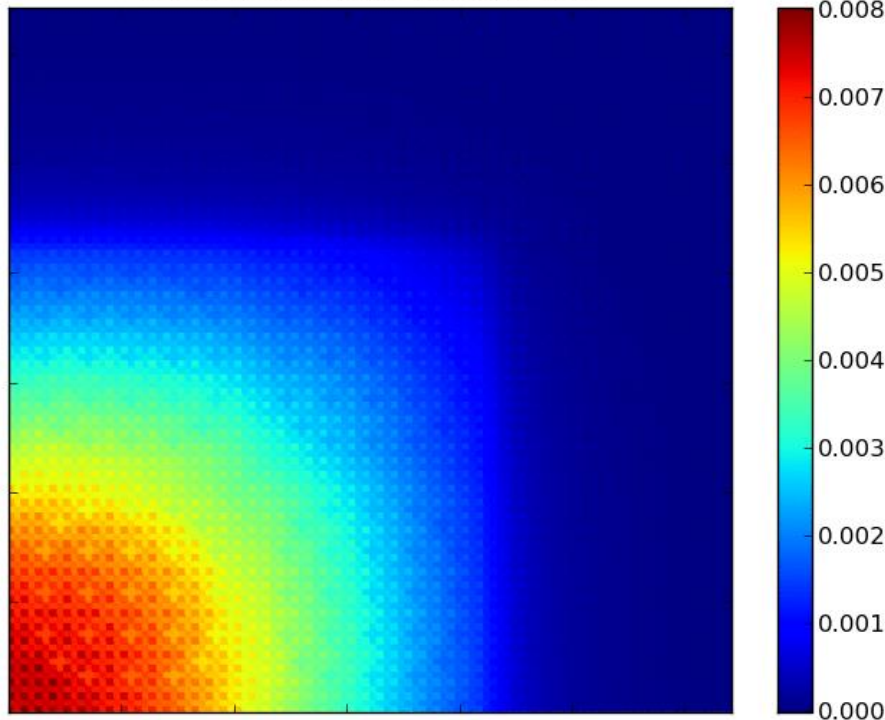


$$k_{\text{eff}} = 0.93287$$

- Notice the local variations in neutron density.
- Different elements can have different neutron densities.

Example of Energy Dependence

C5G7-MOX: Group 1 Scalar Flux





This is Reactor Physics

- It turns out that the nuclear and thermal behavior of the core depends on the natural distribution of neutrons in the core.
- In order to perform any type of nuclear reactor analysis we must be able to determine what the neutron distribution looks like for any given core configuration.
- How to we calculate the natural distribution of neutrons in a reactor core?
 - Not an easy task!
 - Requires us to “think” like a neutron.